

資料リスト

<p>ANL/NDM-121 A Suggested Procedure for Resolving an Anomaly in Least-Squares Data Analysis Known as "Peelle's Pertinent Puzzle" and the General Implications for Nuclear Data Evaluation SEP./1991 Chiba, S. et al.</p>	<p>13888 42 pages</p>
<p>CNDC-0004 CNIC-00412 Communication of Nuclear Data Progress No.3 (1990) 1990 Chinese Nuclear Data Center</p>	<p>13905 65 pages</p>
<p>CNDC-0008 INDC(CPR)-025/L CNIC-00596 Communication of Nuclear Data Progress No.6 (1991) DEC./1991 Chinese Nuclear Data Center</p>	<p>13920 301 pages</p>
<p>ENDF-353 NISTIR 4838 An Evaluation of Kerma in Carbon and the Carbon Cross Sections FEB./1992 Axton, E.J.</p>	<p>13899 138 page</p>
<p>IAEA-NDS-0 Rev.92/5 Index to the IAEA-NDS-Documentation Series MAY /1992 Lemmel, H.D.</p>	<p>13911 15 page</p>
<p>IAEA-NDS-100 Rev.4 ENDF/B-6 the U.S. Evaluated Nuclear Data Library for Neutron Reaction Data JUN./1992 Lemmel, H.D.</p>	<p>13915 26 page</p>
<p>IAEA-NDS-102 Fission-product Yield and Fission-neutron Data for Minor Actinides APR./1992 Lemmel, H.D.</p>	<p>13916 3 page</p>
<p>IAEA-NDS-105 ENDF/B-6 Charged-particle Sublibraries MAY /1992 Lemmel, H.D.</p>	<p>13917 1 page</p>

IAEA-NDS-106	13918
ENDF/B-6 Fission-product Yield Sublibraries	
MAY /1992	
Lemmel, H.D.	2 page
IAEA-NDS-107 Rev.6	13919
Index to BROND-2, CENDL-2, ENDF/B-6, JEF-1, JENDL-3	
JUN./1992	
Lemmel, H.D.	14 page
IAEA-NDS-131 Rev.0	13908
INDEXENDF a PC Code for Indexing Nuclear Data Files in ENDF-6	
Format	
JAN./1992	
Silva, Orion de O. et al.	8 pages
IAEA-NDS-146 Rev.1	13909
POT AUS, A PC code for calculating the stopping power and	
range data of various beams in various materials, by F.E.	
Chukreev	
FEB./1992	
Lemmel, H.D.	1 page
IAEA-NDS-147 Rev.1	13910
Nuclear Masses (RNPL-A), Data File by P. Moeller	
FEB./1992	
Muir, D.W.	13 page
IAEA-NDS-29 Rev.4	13907
ENDF Utility Codes Version 6.7	
MAR./1992	
(Ed.), P.K. McLaughlin	80 page
IAEA-NDS-39 Rev.7	13906
The 1992 ENDF Pre-processing Codes	
FEB./1992	
Cullen, D.E.	150 pages
IAEA-NDS-61 Rev.1	13913
CENDL-2 the Chinese Evaluated Nuclear Data Library for	
Neutron Reaction Data	
JUN./1992	
Lemmel, H.D.	14 page
IAEA-NDS-7 Rev.92/7	13912
Index of Nuclear Data Libraries Available from the IAEA	
Nuclear Data Section	
JUL./1992	
Lemmel, H.D.	29 page
IAEA-NDS-76	13898

ENDF-6 Formats Manual Version of Oct. 1991 JAN./1992 Rose, P.F. et al.	506 pages
IAEA-NDS-90 Rev.5 BROND-2 USSR Evaluated Neutron Data Library JUN./1992 Lemmel, H.D.	13814 15 page
INDC(CCP)-345/G Problems in Atomic Science and Technology, Nuclear Constants, 2 1991 Blokhin, A.I. et al.	13882 130 page
INDC(CCP)-346/G Problems in Atomic Science and Technology, Nuclear Constants, 3 1991	13893 235 pages
INDC(CCP)-347/G Problem in Atomic Science and Technology, Nuclear Constants, 4 (in Russian) 1991	13896 125 pages
INDC(CPR)-025/L CNIC-00596 CNDC-0008 Communication of Nuclear Data Progress DEC./1991 Chinese Nuclear Data Center	13920 301 pages
INDC(CUB)-005/G Can be Achieved a More Accuracy in Neutron Cross Section Calculation at Low Energies? MAR./1992 Cabezas, R. et al.	13904 25 pages
INDC(NDS)-244/M9 Sixth Meeting of the IFRC Subcommittee on Atomic and Molecular Data for Fusion (Summary Report) MAR./1991 Janev, R.K.	13903 71 pages
INDC(NDS)-256/Special Status of Thorium Cycle Nuclear Data Evaluations: Comparison of Cross-Section Line Shapes of JENDL-3 and ENDF/B-VI Files for ^{230}Th , ^{232}Th , ^{231}Pa , ^{233}Pa , ^{232}U , ^{233}U and ^{234}U FEB./1992 Ganesan, S. et al.	13900 138 pages

INDC(NDS)-257/N8	13890
IAEA Advisory Group Meeting on "Atomic and Molecular Data for Fusion Plasma Impurities" Vienna, September 25-27, 1991 FEB./1992	
Janev, R.K.	42 pages
INDC(NDS)-259/N7	13891
IAEA Advisory Group Meeting on "Technical Aspects of Atomic and Molecular Data Processing and Exchange" (10th Meeting of A+M Data Centres and ALADDIN Network) Vienna, 23 and 24 September, 1991 FEB./1992	
Janev, R.K.	59 pages
INDC(NDS)-262/G, NC	13902
11th IAEA Consultants' Meeting of the Nuclear Reaction Data Center Obninsk, 7-11 October 1991 MAR./1992	
Lemmel, H.D.	116 page
ISSN 1019-0643	13897
Comparison Calculation of a Large Sodium-Cooled Fast Breeder Reactor Using the Cell Code MICROX-2 in Connection with ENDF/B-VI and JEF-1.1 Neutron Data FEB./1992	
Sandro Pelloni	43 pages
NEANDC-300*U*	13894
Review of Fission Product Yields and Delayed Neutron Data for the Actinides Np-237, Pu-242, Am-242m, Am-243, Cm-243 and Cm-245 JUL./1991	
Mills, R.W.	91 page
ORNL/CSD/TM-266	13889
Actinide Nuclear Data for Reactor Physics Calculations JUL./1991	
Brady, M.C. et al.	103 page