

資料紹介(そのII)

(以下は非公式に資料を紹介するものでありますから、内容の確認、引用の可否、引用の仕方など全て原典にもどって御判断下さい。)

List of Papers submitted to Conference on Nuclear
Cross Sections and Technology (Washington, March
1975)

標記の国際会議で発表された invited 及び contributed papers のタイトルと著者を紹介します。何れ近いうちに proceedings が本の形で刊行されることと思いますが、information をできるだけ早く多くの層の方にお知らせする意味でリストを示しました。この会議の概要については、日本原子力学会誌 17, No.7 (1975) にある山室信弘・相山一典・浅見明3氏の解説記事から知ることができます。また、paper すべての abstract が Bull. Am. Phys. Soc. II 20, No.2 (1975) に掲載されておりますので、併せ参照していただければ、かなりの内容をつかむことができます。

(編集者)

CONFERENCE ON NUCLEAR CROSS SECTIONS AND TECHNOLOGY, Washington, D. C.
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- AA 2. Neutron Cross-Section Needs.
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- AA 3. The Light Water Reactor Industry----Nuclear Data Needs.
V.O. UOTINEN, J.D. ROBERTSON, and J.S. TULENKO, (Babcock & Wilcox).
- AA 4. Radioactive-Nuclede Decay Data in Science and Technology.
C.W. REICH and R.G. HELMER, (Idaho Naitonal Engineering Laboratory, Aerojet Nuclear Company).
- AA 5. Radioactive Decay Heat Analyses.
R.E. SCHENTER and F. SCHMITTROTH, (Hansford Engineering Development Laboratory).
- AA 6. Accuracy of Fission Product Energy Release Calculations at Short Times After Shutdown.
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- BA 1. Significance of Nuclear Data on Neutron Monitoring of an LMFBR.
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- BA 2. Fast Reactor Safety.
R. AVERY, Argonne National Laboratory.
- BA 3. Nuclear Data and Fast Reactor Safety.
H. KUSTERS, INR, (Kernforschung, Karlsruhe).
- BA 4. After PHENIX, What Is the Importance of Nuclear Data Programs for Fast Breeder Reactor Development?
J.Y. BARRE and J. BOUCHARD, (Centre d'Etude Nucleaires, Cadarache, France).

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- BB 1. NE-213 Neutron Spectrometry System for Measurements to 15 MeV.
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- BB 2. Absolute Calibration of Neutron Detectors in the 10-30 MeV Energy Range.
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- BB 3. A Thick Target Measurement Technique for Determining Nuclear Reaction Rates.
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- BB 4. A Black Detector for 250 keV-1000 keV Neutrons.
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- BB 5. Detector Calibration with an Associated Particle Apparatus.
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- BB 6. Use of Gas Proportional Counters for Neutron Flux Monitors at the NBS Linac.
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- BB 7. Fission Cross Section Measurements on Short Lived Alpha Emitters.
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- BB 8. Systematic Discrepancy in Photoneutron Cross Sections for Medium and Heavy Nuclei.
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- BB 9. The 2 keV Filtered Beam Facility at the NBS Reactor.
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- BB10. The Rensselaer Intense Neutron Spectrometer.
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- BB11. A Modular Minicomputer Multiparameter Data Gathering and Virtual Memory Operating System for the NBS Neutron Standards Program.
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- BB12. TUNL Fast Neutron Cross Section Facility.
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- BB13. A Facility for Studying Neutron-Induced Charged Particle Reactions.
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- BB14. After-Pulse Suppression for 8850 and 8854 Photomultipliers.
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- BB17. A Useful Method for Spin and $\langle \Gamma \gamma \rangle$ Determination Applied to Tm-169 and Au-197.
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- BB18. A 25-keV Neutron Beam Facility at NBS.
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- CA 2. Measurement, Analysis, and Implications of the Fission Cross Sections of the Important Fissionable Isotopes.
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- CA 3. Neutron Capture Cross Section Measurement Techniques.
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- CA 4. Nuclear Models and Data for Gamma-Ray Production.
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- CB 1. Integral Measurements to Test Shielding Cross Sections.
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- CB 2. Evaluation, uncertainty estimation and adjustment of capture cross sections for fission product nuclei.
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- CB 3. Integral Test of Cross Sections Using Neutron Leakage Spectra from Spheres of Beryllium, Niobium, Iron, and Polyethylene.
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- CB 4. Uncertainties and Correlations in Evaluated Data Sets Induced by Use of Standard Cross Sections.
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- CB 5. Shielding Benchmark Experiments and Sensitivity Studies in Progress at some European Laboratories.
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- CB 6. Assessment of Neutron Group Constants for Iron and Stainless Steel through Measurements and Analyses of Energy and Space Distributions of Neutrons in Test Assemblies.
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- CB 7. ENDF/B-IV Dosimetry Cross Section File Benchmark Neutron Spectral Uncertainties.
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- CB 8. Spectral Effects Related to Integral Testing of Neutron Dosimetry Data.
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- CB 9. Comparison of Measured and Calculated Reactivities in CFRMF.
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- CB10. Fission Product Gamma-Ray and Photoneutron Spectra.
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- DA 1. Safeguards Against Theft or Diversion of Nuclear Materials.
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- DA 2. Fission Theory and Actinide Fission Data.
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- DA 3. Nuclear Data for Actinide Recycle.
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- DA 4. n,f Cross Sections for Exotic Actinides,
J.B. WILHELMY, H.C. BRITT, A. GAVRON, E. KONECNY, J. WEBER, (Los Alamos Scientific Laboratory,)
- DA 5. The ^{233}U - ^{232}Th Reactor as a Burner for Actinide Wastes.
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- DA 6. A Consistent Set of Transplutonium Multigroup Cross Sections,
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- DA 7. Measurement of the Neutron Capture Cross Sections of the Actinides.
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- DB 1. Measurements of the $^{10}\text{B}(n,\alpha)$ and $^6\text{Li}(n,\alpha)$ Cross Sections for Neutron Energies from 3 to 1500 keV.
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- DB 2. An Absolute Measurement of the $^6\text{Li}(n,\alpha)$ Cross Section at 964 keV.
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- DB 3. Angular Anisotropy in the $^6\text{Li}(n,\alpha)^3\text{H}$ Reaction at 25 keV.
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- DB 4. Neutron Total Cross Section of ${}^6\text{Li}$ from 100 eV to 3 MeV.
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- DB 5. Observation and Analysis of Elastic Neutron Scattering from ${}^{12}\text{C}$.
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- DB 6. Fission Spectrum Neutrons for Cross Section Validation and Neutron Flux Transfer.
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- DB 7. Fundamental Integral Cross Section Ratio Measurements in the Thermal-Neutron Induced Uranium-235 Fission Neutron Spectrum.
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- DB 8. Interlaboratory Comparison of Absolute Fission Rate and Uranium-238 Capture Rate Measurements in the Mol-22 Secondary Intermediate-Energy Standard Neutron Field.
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- DB 9. Manganese Bath Systematic Effects in Measurements of Nu-Bar and Eta,
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- DB10. Absolute U-235 Fission Cross Section for Cf-252 Spontaneous Fission Neutrons.
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- DB13. Absolute Neutron Flux Determination in Fast Neutron Spectra.
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- EA 2. World Values of the Thermal Parameters of the Fissile Isotopes.
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- EA 3. Neutron Cross Section Standards and Flux Determinations above Thermal Energies.
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- EA 4. R-Matrix Analysis of the Light Element Standards.
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- EB 2. Recent Evaluation for the German Nuclear Data Library KEDAK-3.
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- EB 3. Description of the ENDF/B-IV Silicon Evaluation.
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- EB 4. Evaluation of Fission Product Nuclear Data for 28 Important Nuclides.
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- EB 5. Evaluated Decay-Scheme Data for the ILRR Program.
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- EB 6. Development of a Two-Step Hauser-Feshbach Code with Precompound Decays
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- EB 7. Neutron Cross Sections and Their Uncertainties Obtained from Nuclear
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- EB 8. Level Density Calculation for Deformed Nuclei.
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- EB 9. Neutron Cross Sections of Ni-59.
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- EB10. Statistical Estimation of Physical Quantities in Thermal-and Fast-
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- EB11. Theoretical Estimates of (n, γ) Cross Sections for 6-20 MeV Neutrons.
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- EB12. Neutron Cross Section Calculations for Energies, 0-4 MeV.
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- EB13. Reaction Mechanisms in the High Energy Tail of the 14 MeV Fe56(n,n')
-Process.,
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- EB14. Calculations of (n, α) Rates for Iron Group Materials.
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- EB15. Parametric Fit of the Total Cross Section of ⁴⁵Sc,
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- EB16. Neutron Capture Mechanism in Light and Closed Shell Nuclides.
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- FA 2. Helium Production in Reactor Materials.
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- FA 3. Fast Reactor Fission Yields for ^{233}U , ^{235}U , ^{238}U , ^{239}Pu , and Recommendations for the Determination of Burnup on FBR Mixed Oxide Fuels.
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- FA 7. Burnup Calculations for the KWO-Reactor.
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- FA13. Neutron Energy Spectrum Controlled Blanket for Fast Breeder Reactor.
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- FA15. A Comparison of Air-Over-Ground Transport Calculations Using Different Cross Sections.
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- FA16. The Sensitivity of Neutron Air Transport to Nitrogen Cross Section Uncertainties.
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- FA17. Monte Carlo Studies of the Effect of Cross Section Characteristics on Fast Neutron Penetration in Iron.
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- FA18. Neutron-Coupled Gamma-Ray Transport Cross Sections for Shielding Analysis of Gas-Cooled Fast Breeder Reactors.
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- FA19. Cross Section Preparation for the Continuous-Energy Monte Carlo Code VIM.
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- FA20. A Comparison of VIM and MC^2-2 —Two Detailed Solutions of the Neutron Slowing-Down Problem.
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- FA21. Decay Heat Analysis for an LMFBR Fuel Assembly Using ENDF/B-IV Data.
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- FA22. A Two Dimensional Cross Section Sensitivity Analysis of a Concrete Shield.
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- FA23. GCFR Benchmarks: Experiments and Analysis.
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- FB 2. Energy-Dependent Pion Mean Free Path Length for Star Formation.
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- FB 3. Spectrum and Shielding Measurements and Calculations of Neutrons Produced by 800 MeV Protons.
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- FB 4. Nuclear Data for Assessment of Activation of Scintillator Materials During Spaceflight.
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- FB 5. Proton Scattering for Analysis of Atmospheric Particulate Matter.
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- FB 6. Use of Elastic Scattering Cross Section Anomalies for Depth Profiling Helium and Hydrogen Isotopes in Solids.
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- FB 7. Spallation Cross Sections and the LAMPF MAMPF Medical Radioisotope Program.
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- FB 8. Feasibility of Neutron-Gamma Techniques for Field Analysis of Fresh Concrete.
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- FB 9. Cross Section Requirements for Industrial Gauging Applications
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- FB10. Li, Be and B Production in Proton-Induced Reactions: Implications for Astrophysics and Space Radiation Effects.
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- FB11. Long Lived Isotope Production Cross Sections from Proton Bombardment of Rhenium.
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- FB12. A Need for (p,n) Cross Sections for Selected Targets at Lower Energies.
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- GA 1. Medical Uses of Nuclear Data.
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- GA 2. Medical Use of Fast Neutrons in Radiotherapy and Radiography.
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- GA 3. Biomedical Radiation Transport Calculations as an Application of Nuclear Data .
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- GA 4. Geochemical Mapping of the Moon by Orbital Gamma Ray Spectroscopy.
ROBERT C. REEDY, (Los Alamos Scientific Laboratory).

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- GB 1. ^{235}U Fission Cross Section Measurement from 1 keV to 1 MeV.
J.B. CZIRR and G.S. SIDHU, (Lawrence Livermore Lab)..
- GB 2. The Average Number of Prompt Neutrons, $\bar{\nu}_p$, from Neutron Induced Fission of ^{235}U between 0.2 and 1.4 MeV.
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- GB 3. Monte Carlo Analysis of Monoenergetic Measurement of the .025 eV Eta for ^{233}U and ^{235}U .
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- GB 4. Monte Carlo Analysis of Thermal Spectrum Averaged Measurements of Eta of ^{233}U and ^{235}U .
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- GB 5. Subthreshold Fission in $^{240}\text{Pu}+n$.
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- GB 6. Measurement of the ^{239}Pu Fission Cross-section in the Energy Range from 1 keV to 1 MeV.
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- GB 7. A Measurement of the $^{238}\text{U}/^{235}\text{U}$ Fission Cross-section Ratio .
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- GB 8. Precision Measurements of Fission Neutron Spectra of ^{235}U , ^{238}U and ^{239}Pu .
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- GB 9. Measurement of the Energy Released as Gamma Rays during Radioactive Decay of ^{235}U Fission Products.
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- GB10. Spin Determination of Resonances in ^{235}U .
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