

資料リスト

I. EANDC Documents

(Can)-47L	Progress Report CANADA W. G. Cross (also INDC(Can)-13/L)	.....15 pages
(UK)-154AL	Nuclear Data Requirements for the Reactor Programme in the United Kingdom - March 1973 A. L. Pope (also AEEW-M 1249, INDC(UK)-22G)	...38 "
(UK)-159AL	Review of Fission Product Yield Data for Fast Neutron Fission J. G. Cuninghame (also AERE-R 7548)	.....89 "
(US)-183/L	Neutron Cross Sections Vol. 1, Resonance Parameters S. G. Mughabghab & D. I. Garber (also BNL-325(3rd ed. Vol. I), INDC(USA)-58/L)	..... 758 "
(USA)-187A	The Minutes of the USNDC Meeting 28 - 29 November 1973, Argonne National Laboratory (also INDC(US)-63G, USNDC-10/A)	..... 97 "

II. INDC Documents

-12/L	Official Minutes of the Fifth INDC Meeting R. Joly	..... 177 "
-13/L	Compendium of Committee Regulations of the International Nuclear Data Committee	..... 11 "
(RUM)-4/L	Multilevel Calculation of <sup>235</sup> U Neutron Fission Cross- Section I. M. Mihailscu et al.	..... 9 "
(SEC)-38/U	WRENDA 74 World Request List for Nuclear Data Measurements Fission Reactor Programmes	..... 114 "
(SWT)-5/L	Gamma-Ray Standards J. Kern	..... 366 "
(SWT)-6/G	The Precision Determination of Some $\gamma$ -Ray Energies Using the Fribourg Curved Crystal Spectrometer W. Beer & Jean Kern	..... 16 "

III. Other Documents

AEEW-M1208	The Current Edition of the Main Tape NDL-1 of the UK Nuclear Data Library - March 1973 A. L. Pope	..... 10 "
AEEW-M1234	Thermal Average, Resonance Integral and Fission- Spectrum Average Neutron Capture Cross-Sections of Nuclides with Z = 30 to 68 A. L. Pope & J. S. Story	..... 51 "

AEEW-M1242	One Group Fission Product Capture Cross Sections As Used in Fast Reactor Calculations C. J. Dean .....	8 pages	
ANL/NDM-3	Neutron Scattering From Titanium; Compound and Direct Effects E. Barnard et al. ....	38	"
DP-1327	CUDS: A Computer Program for Processing Cumulative Data Statistics L. G. Anderson & C. W. Lewis .....	42	"
LA-5291-PR	Nuclear Analysis Research and Development Program Status Report .....	22	"
RCN-197	Post Irradiation Examination of Fast Reactor Fuel Pins Irradiated in a Sodium Environment up to about 55 Mwd/kg UO <sub>2</sub> H. Kwast .....	105	"