

資料リスト

I. INDC Documents

7/U	Proceedings of the Topical Conference on "Neutron Induced Fission" held during the 4th INDC Meeting	162 pages
(ARG)-2/G	Neutron Activation Resonance Integral of ^{74}Ge and ^{76}Ge and Evaluation of ^{74}Ge keV Neutron Radiative Capture Cross Section and Resonance Integral M. D. Ricabarra et al.	29 "
(IAE)-2/G	Information System for Nuclear Materials Assay Techniques - Final Report on Work Performed for the International Atomic Energy Agency, Vienna, Austria (Jan. 1, 1971 - Jan. 1, 1972)	
(CCP)-18/L	Bulletine of the Nuclear Data Centre, 7 (Suppl. 1)	
(CCP)-20/L	Radiative Capture of Neutrons by the ^{232}Th Nucleus in the Energy Range 0.01 - 15 MeV A. N. Davletshin et al.	29 "
(CCP)-21/L	Absolute Measurements of α for Uranium-235 and Plutonium-239 in the 10 keV - 1 MeV Neutron Energy Range V. N. Kononov et al.	36 "
(CCP)-23/G	Automation of the Procedure for Checking Information Contained in the Library of Evaluated Nuclear Data; the Pososhok" Programme. V. E. Kolesov et al.	38 "
(CCP)-24/G	Format of the Evaluated Nuclear Data Library for Reactor Calculations V. E. Kolesov et al.	57 "
(USA)-37/G	Fission-Neutron Spectra; Perspective and Suggestion - Prepared for the IAEA Consultants Meeting, Aug. 25, 1971 - A. B. Smith	
(NDS)-40/U	International Cooperation in the Field of Nuclear Data L. Hjörne & J. J. Schmidt	19 "
(NDS)-41	Report on the 7th 4-Centre Meeting, BNL, 25-29, Oct. 1971 V. May	81 "

II. Other Documents

AWRE 066/71	Elastic and Inelastic Scattering of Neutrons in the Energy Range 1.0 to 5.0 MeV by Natural Niobium R. E. Coles	33 "
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RCN-122	STEK, the Fast-Thermal Coupled Facility of RCN at Petten		
RCN-150	Determination of Effective Cross Sections of Fission Products for Fast Breeders in the Fast-Thermal Coupled Critical Reactor STEK		
Neutron Scattering Newsletter	ANL-9		
	i) Total and Elastic Scattering Neutron Cross Sections of ^{239}Pu		
	A. Smith et al.	17	pages
	ii) The Total Neutron Cross Section of Lithium-6 from 100 to 1500 keV		
	J. W. Meadows et al.	13	"
MLM-1798	Determination of Plutonium in Reactor Feed Material		
	F. A. O'hara et al.	44	"
WASH-1170	Bibliography of Non-Destructive Assay Methods for Nuclear Material Safeguards, Feb. 28, 1972		
	R. R. Edwards	87	"
BNL-400 (3rd ed. Vol. 2)	Angular Distriubtions in Neutron-Induced Reactions Vol. II, Z=21 to 94		
	D. I. Garber et al. (also EANDC(US)-138"U")	680	"
LA-4883-PR	Nuclear Analysis Research and Development Program Status Report, Sept.-Dec. 1971	16	"
UCRL-50400 (Vol. 10)	Tabulated Experimental Data for Neutron-Induced Reactions		
	S. T. Perkins, et al.	8	"
UNISIST	Paris 4-8 Oct. 1971 Intergovernmental Conference for the Establishment of a World Science Information System, Final Report		
AERE-M2497	The Simultaneous Evaluation of the Fission Cross-Sections of U-235, Pu-239 and U-238 and the Capture Cross-Section of U-238 in the energy range 100 eV to 20 MeV		
EANDC(E)-143L	An Evaluation for Cross Sections of the Reaction $\text{D(d,n)}^3\text{He}$		
	H. Liskien & A. Paulsen	28	"