Estimate of Photonuclear Reaction in a Medical Linear Accelerator Using a Water-Equivalent Phantom

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In recent radiotherapy techniques, neutrons are generated when the incident photon energy is higher than the threshold of (γ, n) reaction. Reactions occur in the various target materials, as well as in the flattening filters and collimators comprising the head structure of the electron linac. The resulting secondary radiation may lead to secondary cancer in patients because of the increased radiation dose. We measured the secondary neutrons generated in the linac head, used for radiation therapy at 10 and 15 MV using a water-equivalent phantom, gold foil, and solid state track detectors. We calculated the neutron distribution in a model of the same situation using the PHITS code. In Monte Carlo calculations, the scattering of thermal neutrons in the water phantom was confirmed. The contribution from these neutrons was 0.1% or less of that from X-rays at 10 MV. Because the cross-section of the photonuclear reaction increased in the high-energy linac, further examination is required. For 15 MV X-rays, the amount of neutrons was 10 times higher than for 10 MV X-rays. The PHITS code was created to simulate a photonuclear reaction and is therefore suitable for calculations involving these reactions in radioactive materials.

KEYWORDS: photonuclear reaction, medical linear accelerator, Monte Carlo, radiotherapy

I. Introduction

In radiation therapy facilities using medical electron linear accelerators, neutrons are generated by photonuclear reactions occurring between X-rays and the atomic nuclei in various materials, depending on the accelerator energy.1,2) The relative biological effectiveness of a neutron varies according to its energy, and there is a risk of tissue damage and secondary cancer in patients subjected to neutron exposure.3) The treatment of pediatric tumors has come under critical evaluation, because younger patients have a longer life expectancy.4-6) In addition, secondary radiation may lead to radioactivation in the treatment room. Therefore, the evaluation of neutron dose is important in the medical field.

This research employed Monte Carlo calculations using a detailed structural model of the linac head. The simulated neutron distribution and energy spectrum of the linac apparatus represented radiation therapy using 10 and 15 MV X-rays. MV stands for Mega-volt. It is a unit of electric potential. Multiple detectors, including an activation detector and CR-39 in a water-equivalent phantom were used to measure the patient body distribution and the radiation dose from secondary neutrons generated in the water-equivalent phantom.

II. Material and Methods

1. Experimental Configuration

The experiment employed a linac as follows: EXL-15SP (Mitsubishi Electric) for 10-MV and CLINAC iX (Varian Medical Systems) for 15-MV X-rays. A tough water phantom (Kyoto Kagaku) was used as a water-equivalent phantom. Given that the source-surface distance (SSD) was 90 cm, four 5 cm-thick, 30 cm × 30 cm phantoms (for a total thickness of 20 cm) were arranged to simulate a human body.

PHITS code (ver. 2.24) was used. LA150 was used as nuclear data library.8) In addition, secondary radiation may lead to radioactivation in the treatment room. Therefore, the evaluation of neutron dose is important in the medical field.

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Fig. 1 Simulated geometry of the linac head and water equivalent phantom.

Fig. 2 Experimental setup of linac head and water equivalent phantom. Glass dosimeters and CR-39s were positioned at the marked points.

Fig. 3 Experimental setup of gold foils in the phantom.

III. Results

1. Monte Carlo Simulation

The energy spectra of photons, electrons, and neutrons at the head entrance and in the water-equivalent phantom are shown in Fig. 4. The peak energy was 1 MeV. At 15 MV, the neutron dose was four orders of magnitude lower than that of the photon. At 10 MV, the neutron dose was five orders of magnitude lower than that of the photon. Fast neutron generation in the equipment was confirmed, and the serialization of these neutrons in the phantom was observed.

The photon depth doses along the central axis of 10 and 15 MV calculated by PHITS and measured by ion chamber are shown in Fig. 5. The dose distribution is normalized to the maximum dose. Measured and calculated values are similar. Photon and neutron profiles at SCD = 100 cm in a 15 MV linac calculated by PHITS are shown in Fig. 6. The shapes of profiles were different between photon and neutron. The photon and neutron flux at SCD = 95 and 100 cm, calculated using PHITS, are shown in Table 1. Ratios of the neutron to photon are about $10^{-3}$.

2. Photon and Neutron Dose Measurement by Glass Dosimeter and Solid State Track Detectors

The 1 cm dose equivalents of photons and neutrons at SCD = 95 and 100 cm, as measured by glass dosimeter and CR-39, are shown in Table 2. The values were normalized so that the photon radiation dose was 100 monitor units.


As shown in Fig. 2, glass dosimeters and solid state track detectors (Chiyoda Technol Co.) were placed at three locations within the phantom, at source-chamber distances (SCD) of 95 and 100 cm on the beam axis, and at a SCD of 100 cm with a 10 cm lateral displacement from the beam axis. Using a field size of $10 \text{ cm} \times 10 \text{ cm}$, the dosimeters and detectors were irradiated with 10 and 15 MV X-rays from a 90-degree gantry angle, and the photon and neutron doses were measured.


As shown in Fig. 3, gold foil activation detectors ($\phi 10 \text{ mm} \times 0.05 \text{ mm thick}$) were arranged within the phantom from the central beam axis to the direction of Gun-Target and the direction of Right-Left at the position of SCD = 90 cm, 95 cm, 100 cm, 105 cm, and 110 cm, respectively at 5 cm intervals, both along the beam axis and in the lateral and vertical directions. Gold foils are commonly used to measure neutron fluence and energy spectrum, but the use of this many foils hampered the measurement of radioactivity. Therefore, the imaging plate (IP) was adapted for this measurement. The IP had a high sensitivity and a wide dynamic range for radioactive intensity, and provided two-dimensional information. Because the relative intensity of radioactivity was accurately obtained from many foil detectors simultaneously, half-life correction was unnecessary. Under a $10 \text{ cm} \times 10 \text{ cm}$ radiation field, the gold foils after radio-activation were attached to a BAS-MS (Fujifilm) to obtain images every 24 hours using an FLA-7000 (Fujifilm) CR readout device. The pixel values at the position of the gold foil on the image were measured as a relative radioactivity, and the changes over time were analyzed. One sample was measured to obtain the energy spectrum of the activated gold foil using a GMX-10190-P (SEIKO EG&G) HP-Ge semiconductor detector and a MCA600 (Laboratory Equipment) pulse-height discriminator.
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Fig. 4 Energy spectra of photons, electrons, and neutrons in the Clinac iX (15 MV), (a) at the linac head entrance, and (b) in the water-equivalent phantom.

(MU). The neutron dose was approximately three orders of magnitude smaller than the photon dose at 15 MV, and about four orders of magnitude smaller than the photon dose at 10 MV.

3. Neutron Distribution Measured by Gold Foils

A $^{198}$Au energy peak (411 keV) and $^{196}$Au energy peaks (335, 354, and 426 keV) were detected in the measurements of the activated gold foils using the HP-Ge semiconductor detector. Presumably, gold foils were activated with both the $^{197}$Au ($n, \gamma$)$^{198}$Au reaction and the $^{197}$Au ($\gamma, n$)$^{196}$Au reaction. When these nuclides are mixed, it is necessary to divide the PSL (photo-stimulated luminescence) intensity of the IP into $^{198}$Au and $^{196}$Au components. By adopting a least-squares method for the separation of PSL intensity of the IP over time, the $^{198}$Au component was calculated.

The off-axis ratio of the neutron measured by gold foils and the off-axis ratio of the photon measured by the ionization chamber dosimeter in the 10 MV linac are shown in Fig. 7. The neutron fluence had a smooth distribution, whereas the photon dose outside the radiation field is ten times lower than inside. The neutron distributions in the 10 and 15 MV linacs are shown in Fig. 8. There was no significant dependence of the profile on photon energy.

IV. Discussion

The neutron to photon dose ratio was about $10^{-3}$ to $10^{-4}$ for standard irradiation with the electron linac. International Electrotechnical Commission (IEC) recommendations describe four leakage components: (1) downstream of adjustable beam limiting devices; (2) in the patient plane within a radius of 2 m (some states specify 1 m); (3) inside the treatment room; and (4) outside the treatment room. The important factors to the exposure patient are the first two components. Version 2 of IEC 60601–2–1, the standard during the year 2006, recommends that the physical dose downstream of adjustable beam limiting devices due to leakage of primary particles be less than 2% of the prescribed dose. No recommendation is given for the neutron contamination component. Due to the growth of modulated x-ray therapy (IMXT) and the accompanying increase in monitor units and leakage radiation, a proposal for the upcoming
version 3 of the IEC recommendations is that the average value behind adjustable beam limiting devices be limited to 0.75% while retaining the maximum value of 2%. Outside of the projection of the beam limiting device, the existing IEC recommended standard is that the total physical dose from all ionizing radiation should be less than 0.1% of the prescribed dose. No quality factor (QF) or relative biological effectiveness (RBE) is considered but the physical dose from neutrons must be included as part of this 0.1%. In the proposed revision, the non-neutron component is restricted to a maximum of 0.2% and an average of 0.1% while the neutron component is restricted to a maximum of 0.05% and an average of 0.02%. The maximum of leakage ray was confirmed to be close to the value in 15 MV.

In the neutron measurements, there is a possibility of disarranging the place of neutron source by the detector placement and making the measurement difficult. Measuring the high energy photon and neutron energy spectra is difficult. Monte Carlo simulation can calculate energy spectrum with ease. The PHITS code was designed to simulate photonuclear reactions, and can be used to estimate the reactions in radioactive materials. Because the probability of a photonuclear reaction is low, lengthy simulations were required. Therefore, it is preferable to use parallel computation techniques.

Neutron measurements based on gold foils have a high sensitivity, but are unfortunately also sensitive to photons. As photon usage is prevalent in radiation therapy, neutron measurement based on gold foil can therefore be problematic.

Iron has a photonuclear reaction with a photon threshold value of 12 MeV. Wedge filter used to arrange the shape of the dose distribution is made of steel and the use of filter expectedly increases generation of neutron. IMXT, the use of which is becoming increasingly wide-spread, increases the probability of a direct collision between a photon and a multi leaf collimator (MLC). As tungsten MLC has a lower threshold value for photonuclear reaction than steel, neutrons should be more rapidly generated in facilities using IMXT. The neutron dose measurement under such conditions will be discussed in the future.

V. Conclusion

Neutron doses and distributions in a water-equivalent phantom were calculated by PHITS and measured using gold foils.
foils and solid state track detectors. For complex irradiation, such as with a high energy or in IMXT, it is necessary to carefully determine the neutron dose, even though few neutrons are generated under normal conditions.

References


